STUDY OF A NEW SOURCE FOR DUAL PHOTON ABSORPTIOMETRY OF THE LUMBAR SPINE

STUDY OF A NEW SOURCE FOR DUAL PHOTON ABSORPTIOMETRY OF THE LUMBAR SPINE

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To my dear father

ABSTRACT

Bone disease and metabolic bone disorders are characterized by decrease in bone mineral content (BMC) and the vertebrae are primarily affected in osteoporosis. Thus it is essential to monitor changes in BMC due to disease, growth or therapy. Dual photon absorptiometry (DPA) is as yet the most efficient method for BMC determination in the lumbar vertebrae; most previous investigators have used ¹⁵³Gd which is generally accepted as possessing the ideal dual photon energy combination.

Davis and Webber (1978) developed a method for routine production of 153 Sm which emits photons of nearly the same energy as 153 Gd and suggested its use in DPA. In the present study using 153 Sm, measurements with aluminum standards immersed in water resulted in errors of 4% and 4.5% in accuracy and precision respectively for mass of Al in the range 0.4-1.8 g.cm⁻². These results were obtained using different volumes of water, verifying that bone mineral mass determined by this method is independent of soft tissue content. Experiments with bone mineral phantoms gave values which correlated highly (r=0.97) with results obtained on a commercially available clinical densitometer using 125 I.

Considering that these results are comparable with those obtained earlier using 153 Gd, that 153 Gd is scarcely available and several hundred times the cost, 153 Sm promises to be the potential source for routine clinical measurement of osteopenia in the lumbar spine by DPA. However, further invitro and invivo studies are necessary before it can be used regularly in nuclear medicine departments.

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Nomenclature

| I | Attenuated intensity of the higher energy photon |
|--------------------|--|
| I o | Unattenuated intensity of the higher energy photon |
| I' | Attenuated intensity of the lower energy photon |
| I'o | Unattenuated intensity of the lower energy photon |
| ma | Quantity of absorber a in gcm ⁻² |
| m, b | Quantity of absorber b in gcm ⁻² |
| Ni | True count rate, CPS |
| N _o | Observed count rate, CPS |
| × _d | Thickness of NaI(Tı) detector, cm |
| ×s | Radius of rod source, cm |
| × _{A1} | Thickness of aluminum window, cm |
| η | Absolute photopeak efficiency of the detector |
| τ | Deadtime of the system |
| $^{\mu}a$ | Attenuation coefficient of absorber a at the higher energy, cm^2g^{-1} |
| Ψb | Attenuation coefficient of absorber b at the higher energy, cm^2g^{-1} |
| μ <mark>a</mark> | Attenuation coefficient of absorber a at the lower energy, cm^2g^{-1} |
| μ <mark>'</mark> b | Attenuation coefficient of absorber b at the lower energy, cm^2g^{-1} |
| μd | Linear attenuation coefficient of detector material, cm ⁻¹ |
| ^μ s | Linear attenuation coefficient of acrylic, cm ⁻¹ |
| μAl | Linear attenuation coefficient of aluminum, cm ⁻¹ |

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CHAPTER 1

Introduction

The technique of dual photon attenuation (DPA) has been developed and studied during the last fifteen years. The technique is an extension of the single photon method which is suitable for measuring bone mineral content (BMC) in the peripheral or appendicular bones of the body where a constant soft tissue thickness can be maintained (Fig. 1.1). In the single photon technique monoenergetic photons are attenuated through bone mineral and a uniform soft tissue mass; from the measured intensities through soft tissue and through bone, the mass of bone mineral in units of qcm^{-2} is determined using the exponential law of attenuation. The principle of the dual photon method is as follows: when photons at two different energies in a collimated beam are attenuated by bone mineral and non-uniform thicknesses of soft tissue, it is possible to separate the amounts of these substances due to the differential attenuation at the two photon energies by simultaneously solving the two exponential equations shown in Fig. 1.1. For stationary transmission measurements, this method will result in accurate values for the amount of bone mineral and soft tissue in the path of the photons in an ideal two compartment system. To determine the bone mineral content across a given area, a rectilinear scan procedure is followed with predetermined scan velocities and scan paths. However, corrections to both methods have to be made during in vivo measurements when varying quantities of facty or adipose tissue are present constituting a three



Fig. 1.1: Sketch of single and dual photon methods.

compartment system. In both methods, the transmitted photons are detected by a collimated NaI (T ℓ) detector located at a constant distance from the source.

Generally clinical requirements are for mineral measurements in the central or axial skeleton where bone fractures occur. Unfortunately at such sites constant soft tissue thickness cannot be provided. In these circumstances the dual photon technique is imperative. Its usefulness in the determination of bone mineral content for clinical applications has been well established. Bone disease and metabolic bone disorders are characterized by decrease in BMC [1] and a concurrent loss of bone strength. A high correlation was observed between mineral content and crushing stress at elastic limit in lumbar vertebrae [2]. Krolner and Nielsen [3] have shown in their cross-sectional and longitudinal studies that measurement of lumbar BMC is useful for the diagnosis of spinal osteoporosis. The vertebrae are primarily affected in senile osteoporosis [4] leading to vertebral fractures and hence measurement of spinal BMC is important for diagnosis of osteoporosis.

Most investigators used either the isotope 153 Gd (photons at 44 kev and 100 kev) or combinations of two isotopes such as 241 Am (59.6 kev) and 137 Cs (662 kev) as sources for photons of two different energies. The application of dual photon absorptiometry was first suggested in 1953 by Cameron and Sorenson [5] using the radioactive isotopes 125 I and 241 Am for the determination of BMC in the forearm. They reported an average accuracy of 3% when the quantity of CaCO₃ present in mixtures of CaCO₃ and paraffin was measured. Reed [6]

described the use of the dual sources 241 Am and 137 Cs to determine calcium in bone and to study bone mineralization. Roos et al [7] utilized the same isotope pair in a scanning procedure for measuring bone mineral in the vertebrae. The dual photon technique was also developed by Mazess et al [8] with the radioisotope 153 Gd which has photon emissions around 44 kev and 100 kev.

Hanson [9] modeled the precision of the dual attenuation technique by an expression for the coefficient of variation of bone mineral measurement, to compare the dual photon sources 153 Gd and 125 I, 241 Am. His results showed that for the ratio of bone mineral mass to soft tissue mass of 0.2, 153 Gd was more precise for thick body sections (where bone mineral >1.2 gcm⁻²) and ^{125}I , ^{241}Am was more precise for thin sections (where bone mineral <1.2 gcm^{-2}). In another theoretical analysis, Watt [10] evaluated the minimum Relative Error Function (defined as a direct proportion of the coefficient of variation of bone mineral) to determine the optimum energy combination for different absorber thicknesses and bone mineral fractions. With 15.0 gcm^{-2} total absorber thickness and 0.1 fraction by weight of bone mineral, he obtained the optimum energies for maximum precision to be 40 keV and 400 kev in the dual photon method. Smith et al [11] compared the sources 153 Gd and 241 Am, 137 Cs for soft tissue thickness of 5 gcm⁻² to 30 gcm^{-2} and bone mineral content of 0.6 gcm^{-2} to 1.4 gcm^{-2} which are typically present in normal adults. By minimizing the Comparative Error Function (CEF) which is similar to the REF defined by Watt [10], they found the optimum energy combination to be 43 kev and 180 kev for a bone mineral of 1.0 gcm⁻² and soft tissue thickness of 20 gcm⁻².

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Their results showed that the upper energy level had little effect on CEF and that 153 Gd was superior to 241 Am, 137 Cs which also gave clinically viable results. However, they argued that in view of the irregular availability and greater cost of 153 Gd, the dual source 241 Am, 137 Cs was a practical alternative for spine BMC measurements.

In order to evaluate changes in BMC in the lumbar spine due to disease, growth or therapy, a high accuracy and precision in the measurements is essential. In studies with 153 Gd, Mazess et al [12] obtained an accuracy below 2% with dipotassium hydrogen phosphate standards immersed in water. They found the BMC in normal persons to be uniform along the spine within 3% and in osteoporosis they observed about 70% mineral loss in specific areas of the spine. Wilson and Madsen [13] reported a correlation coefficient of 0.99 in measurements using dipotassium hydrogen phosphate phantoms to simulate spinal bone tissue. Dunn et al [14] also obtained a correlation coefficient of 0.99 between actual and measured mass of ashed bone samples immersed in water and oil. Krolner and Nielsen [15] developed a scanning technique using 153 Gd and achieved invitro accuracy of 1.3% in measurements with bone mineral equivalent solutions.

As indicated by Smith et al [11], for clinical studies of metabolic disorders of bone, the precision of the dual photon method should be around 2%. Kan et al [16] developed an analog device for direct readout of BMC and bone width from dichromatic absorptiometry using 153 Gd and obtained a precision below 2% on human upper arm measurements. Roos and Skoldborn [17] utilized the dual source 241 Am, 137 Cs in

their experiments with patients and phantoms. They obtained a precision of 3% in measurements with phantoms and 3-8% in measurement of the lumbar spine BMC in patients. Corrections for the presence of adipose tissue were made by baseline adjustment of the transmission curve. In studies with 153 Gd. Mazess et al [12] reported a precision of 3% for the BMC of the lumbar spine in normal subjects measured over a one month period. In measurements with dipotassium hydrogen phosphate phantoms, Wilson and Madsen [13] obtained a precision of 1.7% and in measurements on four individuals the precision ranged from 0.7-3.8%. Condon et al [18] used the dual sources 241 Am, 137 Cs in measuring bone mineral in the 2nd, 3rd and 4th lumbar vertebrae and obtained long-term reproducibility of 2% on phantoms and 3% on patients. Krolner and Nielsen [15] achieved in vitro precision of 0.9-2.3% in measurements with bone mineral standards and human lumbar vertebrae over a six month period. The invivo precision in measurements on ten subjects varied from 1.4-2.6%. In another study using 153Gd, Dunn et al [14] reported a precision of 1.8% with bone mineral phantoms and 1.3% for lumbar spine measurements in five volunteers. Tothill et al [19] compared the sources 153 Gd and 241 Am, 137 Cs for the measurement of BMC in the lumbar spine by DPA and concluded that 153Gd was superior with a precision of 1-3% in subject measurements. Riggs et al [20] used the isotope 153Gd and obtained a precision of 2.3% in lumbar spine measurements on five volunteers during a ten month period. They showed that spinal osteoporosis was characterized better by the vertebral bone mineral density rather than the presence of nontraumatic vertebral fractures and they defined a value of 0.965 gcm^{-2} as a threshold below which the

risk for vertebral fracture increased. Their results proved that direct measurements of bone density was essential to assess spinal osteoporosis. Smith et al [11] obtained a precision of 1.1-2.5% with an anthropomorphic phantom using 153 Gd and with a wax phantom their precision was 0.6% using 241 Am, 137 Cs and 0.4% using 153 Gd. The precision of fat estimation with 20 gcm⁻² soft tissue and 15% fat was 1.7% using 153 Gd and 3.8% using 241 Am, 137 Cs.

The isotope 153 Sm emits photons of nearly the same energy as 153 Gd. Davis and Webber [21] developed a method for routine production of 153 Sm in a reactor, showing it to be economical and more easily available than 153 Gd and suggested that 153 Sm could replace 153 Gd in all applications where it has hitherto been used, including dual photon absorptiometry.

¹⁵³Gd has a half-life of 242d and has to be replaced annually.
¹⁵³Sm has a half-life of 46.8h and needs to be replaced every week.
As shown by Smith et al [11], a high activity ¹⁵³Gd source could be used for two years provided the source collimator dimensions were changed to avoid deadtime errors at high activity and to obtain a high enough count rate at low activity. The disadvantage of this will be the change in geometry with a possible introduction of errors into BMC measurements.
Ideally, longitudinal studies spanning several years necessitate absolutely reproducible parameters for useful results. This would be possible with weekly production of ¹⁵³Sm.

The purpose of this preliminary investigation is to study the radioisotope 153 Sm and to evaluate and assess its usefulness in DPA of the lumbar vertebrae.

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CHAPTER 2

Efficiency Calibration of a Thin NaI(Tr) Crystal Detector

2.1 Introduction

The objective of the undermentioned measurements is to determine the absolute intensities of the two groups of photons emitted from the radioisotope source $(^{153}$ Sm) to be used for In-Vivo dual photon transmission measurements of the mineral content of the lumbar spine. 153Sm is produced in the McMaster University nuclear reactor by thermal neutron activation of enriched 152 Sm. Following the negatron beta decay of 153 Sm, the predominant emissions are 103 kev gamma rays and Eu Kxrays (242.5 kev) due to internal conversion. To fully understand the relation between transmission of photons through an object and the elemental composition of the object, it is necessary to know the incident flux at each photon energy. The incident flux for a given source will be determined by the inherent decay characteristics of 153 Sm and the self-absorption of photons by the source. A series of measurements were designed using a $NaI(T_{\ell})$ detector to measure the absolute intensities of the major photon emissions of 153 Sm and to assess the self-absorption in a number of reactor produced 153 Sm sources.

The absolute photopeak efficiency of a scintillation detector is defined as the ratio of the number of gamma rays totally absorbed in the crystal to the number of source gamma rays emitted within the solid angle subtended by the detector at the source. The number of gamma rays totally absorbed in the crystal is given by the photopeak area



Fig. 2.1: Source-detector arrangement.

of the pulse height spectrum obtained on a multichannel analyzer (canberra series 30). The energy range of concern is 10 kev to 130 kev. The detector system consists of a 5.08cm Dia. x 0.3175 cm thick NaI(T₂) crystal with a 0.0254 cm thick aluminum window and a lead collimator 2.54cm I.D. x 5.0 cm long. Standard rod sources of 57 Co, 109 Cd, 133 Ba and 241 Am were used with source to detector distance 5.5 cm in all cases (Fig. 2.1).

2.2 Theoretical Evaluation

The efficiency values for different energies were calculated using the following expression [22]:

$$\eta = [1 - \exp(-\mu_d \mathbf{x}_d)] \cdot \exp(-\mu_s \mathbf{x}_s) \cdot \exp(-\mu_{A\ell} \mathbf{x}_{A\ell})$$

where n is the absolute photopeak efficiency; μ_d , μ_s , $\mu_{A\ell}$ are the total absorption coefficients of the detector material (NaI(T ℓ)), intermediate source attenuating material (Acrylic) and of aluminum respectively; x_d , x_s and $x_{A\ell}$ are the path lengths of the gamma rays through the three corresponding materials. Self absorption in the rod sources ($e^{-\mu_s X_s}$) was considered since the relative absorption at low energies is high. The values of the absorption coefficients used were taken from the tables of Storm and Israel [23]. Since these were given for 10 kev intervals it was important to fit the data to an expression to allow the interpolation of coefficients at intermediate energies. Various polynomial fits of different degrees (using the International Mathematics and Statistics Libraries (IMSL) subroutine RLFOR) failed to give accurate expressions for absorption coefficients of Na, I and A ℓ . The best polynomial fits across the energy range 10-150 kev yielded for the percentage of variation of the estimated models, values as large as 30%. The data were then grouped into energy regions based on the linearity observed over different portions of a log-log plot of absorption coefficients and energy. A general expression of the form

$$\ln \mu = A \ln E + B$$

where μ is in cm² g⁻¹, E is in kev and A and B are constants, was fitted to the data for each energy region. The constants were determined using IMSL subroutine FLONE. The results are given in Table 2.1.

For each element, the value at 122.1 key was obtained by linear interpolation. Censities of 2.694 gcm⁻³ for A_{ℓ} and 3.67 gcm⁻³ for NaI were used. To estimate self-absorption in the rod source, the data given by the manufacturer (New England Nuclear) was used; that is a rod diameter of 1.27 cm and composition of acrylic as 60.04% carbon, 7.97% Hydrogen, 31.99% oxygen with a density of 1.185 gcm^{-3} . As before, the total absorption coefficients of C, H and O were obtained from [23] and fitted as shown in Table 2.1. In each case, the values at energies outside the indicated energy ranges were obtained by linear interpolation. The total absorption coefficients used are shown in Table 2.2 along with the computed efficiencies. A maximum error of 10% is quoted in [23] for the total absorption coefficients. Tables of these values in [24] report an accuracy of 1% to 2% and [25] gives polynomial fits for these values with an error of 1%. Using [25], the total absorption coefficients of $NaI(T_{\ell})$ are computed with a maximum error of 3% and are also shown in Table 2.2 along with the calculated efficiencies.

| ELEMENT | ENERGY REGION (KeV) | А | В | CORR. COEFF. |
|---------|------------------------|--------|--------|--------------|
| I | 8 - 33.17 | - 2.73 | 11.31 | 1.00 |
| | 33.17 - 100 | - 2.04 | 9.81 | 0.99 |
| | 10 - 50 | - 3.09 | 9.79 | 1.00 |
| Na | 50 - 100 | - 1.87 | 4.99 | 0.99 |
| Al | 10 - 100 | - 2.91 | 9.82 | 1.00 |
| Ċ | 10 - 50 | - 2.82 | 7.02 | 0.99 |
| Н | 10 - 50 | 0.66 | - 6.23 | 0.99 |
| 0 | 10 - 60 | - 2.93 | 8.30 | 0.99 |

TABLE 2.1 - Parameters for computing absorption coefficients

| ENERGY (KeV) | M _{NaI} (cm ⁻¹) | μ. Al (cm ⁻¹) | µ Acrylic (cm ⁻¹) | Efficiency (using (23)) | M _{NaI} (cm ⁻¹ from (25)) | Efficiency (using (25)) |
|-----------------|---|---------------------------------|-------------------------------------|----------------------------|---|----------------------------|
| 14.4 | 177.49 | 21.15 | 1.06 | 0.31 | 199.60 | 0.31 |
| 22.2 | 55.70 | 6.16 | 0.31 | 0.71 | 63.13 | 0.71 |
| 31.0 | 21.81 | 2.27 | 0.12 | 0.88 | 21.71 | 0.88 |
| 59.6 | 13.34 | 0.34 | 0.03 | 0.96 | 24.20 | 0.97 |
| 81.0 | 7.13 | 0.14 | 0.03 | 0.88 | 10.90 | 0.95 |
| 88.0 | 6.02 | 0.11 | 0.03 | 0.84 | 8.46 | 0.91 |
| 122.1 | 3.09 | 0.09 | 0.03 | 0.61 | 4.00 | 0.70 |

TABLE 2.2 - Theoretical Efficiency at Different Energies

At very low energies (<25 kev), absorption in the AL window and self-absorption in the rod source is high, resulting in low detection efficiencies. As the photon energy increases, this absorption decreases. At the same time absorption in NaI increases rapidly with an increase in detection efficiency. At even higher energies (>60 kev), absorption in NaI decreases, there is little source self-absorption and absorption in the AL window, leading to lower efficiencies.

2.3 Experimental Determination

The detector is a crystal-photomultiplier integral unit (ser. # HN708174) and was supported with its axis vertical. The photomultiplier tube preamplifer was connected through a DDL Amp. (Canberra Model 1411) to the MCA. A reference pulser (Canberra Model 1407) was also coupled to the input of the amplifier. The pulse amplitude was adjusted so that the pulser peak was located on the MCA display beyond the pulse height spectra cf the rod sources. The pulser was necessary to correct for counting losses which occurred despite deadtime corrections in the timer setting of the MCA. In each case background counts were subtracted. The various gamma ray intensities and the isotope half lives are given in Table 2.3. Since no unified reference is available for these isotopes, it was necessary to use the results of different investigators to compile the data shown in Table 2.3.

Three measurements of detector efficiency were made over a period of four days for each rod source positioned 5.5 cm from the detector face. The results are shown in Table 2.4. Corrections were made for the presence of associated photons in the photopeaks listed

| ISOTOPE | ENERGY (KeV) | INTENSITY (%) | HALF-LIFE | REFERENCE |
|-----------|-----------------|---------------------------|---------------------|------------|
| 57 Co | 14.4 122.1 | 9.38 85.3 | 270d | (26), (27) |
| 109 Cd | 22.2 88.0 | 85.3 3.79 | 453d | (28) |
| 133 Ba | 31.0 81.0 | 95.1 3 ⁴ .7 | 10.7 ⁴ y | (29) |
| 241 Am | 59.6 | 36. 0 | 458y | (26) |

TABLE 2.3 - Intensity and Half-life data for

the radioisotopes used

| ENERGY (KeV) | CPS FROM SPECTRUM AFTER CORRECTION | UNATTENUATED SOURCE ACTIVITY PHOTON/SEC | EFFICIENCY | AV. EFFICIENCY |
|-----------------|---------------------------------------|--|---|-------------------|
| 14.4 | 29.34 28.47 28.03 | 1.52×10^4 1.51×10^4 1.50×10^4 | $\begin{array}{r} 0.27 \pm .02 \\ 0.27 \pm .02 \\ 0.26 \pm .02 \end{array}$ | 0.27 ± .02 |
| 22.2 | 1762.12 1714.36 1692.34 | 24.98×10^{4} 24.90 x 10 ⁴ 24.86 x 10 ⁴ | 0.64 ± .04 0.62 ± .04 0.62 ± .04 | 0.63 <u>+</u> .04 |
| 31.0 | 19.56 20.03 19.80 | $0.19 \times 10^{4} \\ 0.19 \times 10^{4} \\ 0.19 \times 10^{4} \\ 0.19 \times 10^{4}$ | 0.83 ± .06 0.85 ± .06 0.84 ± .06 | 0.84 <u>+</u> .06 |
| 59.6 | 867.20 884.50 849.33 | $6.98 \times 10^{4} \\ 6.98 \times 10^{4} \\ 6.98 \times 10^{4} \\ 6.98 \times 10^{4}$ | 0.92 ± .06 0.93 ± .06 0.90 ± .06 | 0.92 <u>+</u> .06 |
| 81.0 | 7.89 7.98 7.88 | $ \begin{array}{r} 0.07 \times 10^{4} \\ 0.07 \times 10^{4} \\ 0.07 \times 10^{l4} \end{array} $ | 0.87 ± .06 0.88 ± .06 0.87 ± .06 | 0.87 <u>+</u> .06 |
| 88.0 | 118.89 121.57 119.29 | 1.11 x 10^{4} 1.11 x 10^{4} 1.10 x 10^{4} | $\begin{array}{r} 0.82 \pm .06 \\ 0.83 \pm .06 \\ 0.82 \pm .06 \end{array}$ | 0.82 <u>+</u> .06 |
| 122.1 | 987.44 1027.69 963.13 | 13.79×10^{4} 13.72×10^{4} 13.69×10^{4} | $0.55 \pm .04$ $0.57 \pm .04$ $0.54 \pm .04$ | 0.55 <u>+</u> .04 |

TABLE 2.4 - Measured efficiencies at various photon energies

in Table 2.3. These were for 109 Cd the 25 kev k_R x-rays, for 133 Ba the 35 kev $k_{_{\rm \beta}}$ x-rays and the 79.6 kev gamma rays and for $^{57}{\rm Co}$ the 136.5 kev gamma rays. Intensities of these associated photons were also obtained from the corresponding references listed in Table 2.3. No corrections were made for photon backscattering or for scattering from the collimator, because the measurement geometry made backscattering unlikely and scattering from a lead collimator is not favoured for low energy photons. In order to measure iodine escape, a 2 mm thick Cu absorber was placed between the rod source and detector in order to totally absorb the lower energy source emissions (<45 key). An iodine escape peak, of 15.5%, was observed only for the ²⁴¹Am source and correction due to this was made for the 59.6 kev measurement. The absence of promirent iodine escape peaks for the 81, 88 and 122.1 kev photons is possibly due to the rapid increase of mean free path for increasing energies (0.4 mm for 59.6 kev, 0.9 mm for 81 kev, 1.2 mm for 88 key and 2.5 mm for 122.1 key) and also due to low energy resolution of the detector at these energies. Finally, for isotopes with emissions at two energies, measurements were made with the same cu absorber as before. The contribution of the 88 kev photons to the 22.2 kev energy window in the MCA was found to be 3.8%, contribution of the 81 kev photons to the 31 kev window was 16% and that due to 122.1 kev photons to the 14.4 kev window was 0.5%; accordingly, corrections were made for the lower energy measurements.

2.4 Conclusions

The theoretical and experimental values of efficiency are com-



Fig. 2.2: Plots of efficiency vs. energy.

pared in Fig. 2.2 and a good correspondence is observed. The typical error due to counting statistics in the measurements is 1%. The accuracy of the source activities is 5%. Errors arising from variations in positioning of the rod source and from the corrections mentioned earlier, mean that the overall error in the results may be 7%. From these results it is concluded that the detector efficiencies for the 103 kev and 42.5 kev photons of 153 Sm are 0.71 and 0.90 respectively.

CHAPTER 3

Count Rate Characteristics

In this chapter the following five characteristics of the sourcedetector combination are discussed: the nature of the relation between water thickness and attenuation of the 42.5 and 103 kev photons; deadtime effects; photon absorption in the source material; source activity as a function of detector count rate and finally the geometry of the photon beam.

3.1 Exponential Attenuation

The technique of DPA is**applicable to** photon intensities following the exponential law of attenuation. This has been investigated for a range of water thicknesses from 5 cm to 25 cm, for the 42.5 kev and 103 kev photons of ¹⁵³Sm.

Measured quantities ($\sim 26 \text{ mg}$) of ${}^{152}\text{Sm}_20_3$ (98.29%) were inserted into specially constructed graphite capsules shown in fig. 3.1. These capsules were irradiated in the McMaster Nuclear Reactor for a few hours to obtain ${}^{153}\text{Sm}$. The collimated source holder was located on a modified OHIO Nuclear Rectilinear scanner and the NaI(Tr) detector aligned at a distance of about 50 cm from the source. Known volumes of water were taken in a graduated glass cylinder to obtain different thicknesses. Counts in the 42.5 kev and 103 kev photopeaks were registered in the MCA and the contribution from the higher energy to the lower energy region was determined using 4 mm thick copper sheets to absorb the low energy photons. Windows were set to include the total photopeaks in





All dimensions in MM

A Graphite capsule

B Cylindrical graphite insert with central hole

C Threaded graphite cap

D Assembled source capsule

Fig. 3.1: Graphite capsule for 153 Sm source

both the energy channels. The compton contribution to the lower energy channel was found to be 7% of total counts under the higher energy peak. This was obtained from measurements with different ¹⁵³Sm sources and different water thicknesses. The variation was less than 1% showing it to be independent of water thickness. Tables 3.1A, 3.1B and 3.1C indicate for three separate experiments each performed with the same source, the measured count rates for different water thicknesses along with the corrected true count rates. The MCA counts were corrected for decay and as before, a pulser was used concurrently to obtain the true counts. A logarithmic fit using RLONE yielded for the measurements in each experiment, the corresponding exponential equations shown in Table 3. 1 for the 42.5 kev and 103 kev photons. The derived attenuation coefficients for water are also indicated in these tables and the mean values are 0.246 \pm .007 cm⁻¹ at 42.5 kev and 0.170 \pm .005 cm⁻¹ at 103 kev. By fitting the data of Hubbell (1969) for the total attenuation coefficient for water, the values of 0.255 cm^{-1} and 0.169 cm^{-1} were obtained at 42.5 kev and 103 kev respectively. The error reported by Hubbell is 1-2% in his attenuation data and the error in the derived data is 3%. The above results verify that these photon intensities obey the exponential law of attenuation.

3.2 Deadtime

Another common source of error in count rate measurements is due to the deadtime of the system. In this respect two types of deadtimes are recognized - paralyzable and nonparalyzable. The type of response depends on whether or not the deadtime of the system is

| WATER THK. | COUNT TIME | PULSER | COUNTS | OBS. CPS. W, | DECAY CORR. | TRUE CPS, | N |
|------------|----------------------------|----------------|--------|---------------------------------------|-------------------------------|-----------|---------|
| cm, x | SEC. | TRUE | OBS. | 42.5 KeV | 103 KeV | 42.5 Kev | 103 KeV |
| 2.55 | 10 | 600 | 566 | 3577.4 | 4247.7 | 3792.3 | 4502.9 |
| 3.82 | 10 | 600 | 582 | 2607.2 | 3476.6 | 2687.8 | 3584.1 |
| 5.10 | 10 | 600 | 595 | 1934.9 | 2675.8 | 1951.2 | 2698.3 |
| 6.37 | 20 | 1200 | 1196 | 1396.8 | 2160.5 | 1401.5 | 2167.7 |
| 7.65 | 20 | 1200 | 1185 | 1026.7 | 1774.4 | 1039.7 | 1796.9 |
| 8.92 | 20 | 1200 | 1174 | 718.2 | 1438.1 | 734.1 | 1469.9 |
| 10.20 | 30 | 1800 | 1796 | 548.5 | 1173.7 | 549.7 | 1176.3 |
| 11.47 | 30 | 1800 | 1787 | 394.1 | 937.4 | 396.9 | 944.2 |
| 12.75 | 50 | 3000 | 2979 | 283.0 | 759.1 | 285.0 | 764.4 |
| 14.02 | 50 | 3000 | 2983 | 201.2 | 609.1 | 202.3 | 612.6 |
| 15.30 | 90 | 5400 | 5376 | 151.1 | 486.4 | 151.8 | 488.6 |
| 16.57 | 90 | 5400 | 5369 | 112.3 | 400.1 | 112.9 | 402.4 |
| 17.85 | 90 | 5400 | 5374 | 75.1 | 317.5 | 75.5 | 319.0 |
| 19.12 | 200 | 12000 | 11987 | 55.6 | 252.9 | 55.7 | 253.2 |
| 20.40 | 200 | 12000 | 11984 | 42.0 | 208.4 | 42.1 | 208.7 |
| 21.67 | 200 | 12000 | 11997 | 30.2 | 165.4 | 30.2 | 165.4 |
| 22.95 | 300 | 18000 | 17998 | 22.3 | 135.2 | 22.3 | 135.2 |
| 24.22 | 400 | 24000 | 23997 | 17.2 | 107.2 | 17.2 | 107.2 |
| 25.50 | 500 | 30000 | 29996 | 12.8 | 89.8 | 12.8 | 89.8 |
| | <i>m</i> _{42.5} = | 0.250 <u>+</u> | .007 c | ^{m-1} ; ¹ 103 = (| 0.170 <u>+</u> .005 c 250x | em -1 | |

for 42.5 KeV: N = 6925.2e -.250x for 103 KeV: N = 6658.4e -.170x

<u>TABLE 3.1A</u> - Attenuation coefficient for water -1

| WATER THK. | COUNT TIME | PULSER | COUNTS | OBS. CPS. | w/DECAY CORR. | TRUE CPS | N |
|------------|-------------|----------------|--------|-----------|---------------|----------|---------|
| cm, x | SEC. | TRUE | OBS. | 42.5 KeV | 103 KeV | 42.5 KeV | 103 KeV |
| 2.55 | 20 | 1200 | 1193 | 1269.1 | 1946.9 | 1276.5 | 1958.3 |
| 3.82 | 20 | 1200 | 1192 | 890.4 | 1524.9 | 896.4 | 1535.1 |
| 5.1 | 20 | 1200 | 1196 | 631.3 | 1228.8 | 633.4 | 1232.9 |
| 6.37 | 20 | 1200 | 1194 | 452.7 | 990-8 | 454.9 | 995.8 |
| 7.65 | 30 | 1800 | 1792 | 343.7 | 802.2 | 345.2 | 805.8 |
| 8.92 | 30 | 1800 | 1789 | 238.6 | 636.0 | 240.1 | 639.9 |
| 10.2 | 40 | 2400 | 2380 | 174.2 | 519.5 | 175.7 | 523.9 |
| 11.47 | 60 | 3600 | 3589 | 127.6 | 417.4 | 128.0 | 418.7 |
| 12.75 | 60 | 3600 | 3592 | 93.3 | 336.4 | 93.5 | 337.1 |
| 14.02 | - 80 | 4800 | 4798 | 68.3 | 268.9 | 68.3 | 269.0 |
| 15.3 | 80 | 4800 | 4789 | 52.0 | 227.4 | 52.1 | 227.9 |
| 16.57 | 1 00 | 6000 | 5999 | 36.4 | 178.3 | 36.4 | 178.3 |
| 17.85 | 200 | 12000 | 11998 | 25.7 | 143.9 | 25.7 | 143.9 |
| 19.12 | 300 | 18000 | 17996 | 21.1 | 117.5 | 21.1 | 117.5 |
| 20.4 | 300 | 18000 | 17998 | 15.1 | 93.4 | 15.1 | 93.4 |
| 21.67 | 400 | 24000 | 23998 | 11.7 | 76.2 | 11.7 | 76.2 |
| 22.95 | 500 | '30 000 | 29997 | 8.1 | 61.6 | 8.1 | 61.6 |
| 24.22 | 500 | 30000 | 29999 | 6.7 | 50.4 | 6.7 | 50.4 |
| 25.5 | 600 | 36000 | 35996 | 4.8 | 40.4 | 4.8 | 40.4 |

 $M_{1/2.5} = 0.21/2 \pm .007 \text{ cm}^{-1};$ $M_{103} = 0.168 \pm .005 \text{ cm}^{-1}$

for 42.5 KeV: N = 2148.1e^{-.242x}

for 103 KeV: N = 2913.1e -.168x

TABLE 3.1B - Attenuation coefficient for water -2

| WATER THK. | COUNT TIME | PULSER COUNTS | | OBS. CPS. | w/DECAY COR. | TRUE CPS,N | |
|------------|------------|---------------|------|-----------|--------------|------------|---------|
| cm,x | SEC. | TRUE | OBS. | 42.5 KeV | 103 KeV | 42.5 KeV | 103 KeV |
| 5.1 | 10 | 600 | 540 | 6740.0 | 7401.4 | 7488.9 | 8223.8 |
| 7.65 | 10 | 600 | 593 | 3683.0 | 5406.8 | 3726.5 | 5470.6 |
| 10.2 | 10 | 600 | 586 | 1938.3 | 3189.5 | 1984.6 | 3265.7 |
| 12.75 | 10 | 600 | 583 | 963.6 | 2126.0 | 991.7 | 2188.0 |
| 15.3 | 20 | 1200 | 1192 | 540.4 | 1465.8 | 544.0 | 1475.6 |
| 17.85 | 20 | 1200 | 1180 | 310.2 | 917.9 | 315.4 | 933.4 |
| 20.4 | 40 | 2400 | 2363 | 167.6 | 557.1 | 170.2 | 565.8 |
| 22.95 | 60 | 3600 | 3565 | 95.2 | 357.5 | 96.1 | 361.0 |
| 25.5 | 90 | 5400 | 5352 | 42.0 | 269.5 | 42.4 | 271.9 |

$$\mathcal{M}_{42.5} = 0.247 \pm .007 \text{ cm}^{-1}; \quad \mathcal{M}_{103} = 0.171 \pm .005 \text{ cm}^{-1}$$

for 42.5 KeV:N = 24873.8e -.247 x
for 103 KeV:N = 19507.9e -.171 x

<u>TABLE 3.1C</u> - Attenuation coefficient for water -3



Fig. 3.2 : Transmission of 42.5 Kev photons through water


Fig.3.3 : Transmission of 103 Kev photons through water

lengthened by an input event reaching the system while the previous event is being processed. The following mathematical expressions, in which N_i and N_o are the true and observed count rates and τ is the system deadtime [30], describe the two cases.

> paralyzable deadtime: $N_0 = N_i e^{-N_i \tau}$ nonparalyzable deadtime: $N_0 = \frac{N_i}{(1+N_i \tau)}$ or $N_i = \frac{N_0}{(1-N_0 \tau)}$

As reported [30], it is apparent that for a paralyzable system, the observed count rate increases to a maximum and then decreases. At very high true count rates, the observed count rate becomes zero and the system is totally paralyzed. Also, two values of the true count rate (one very much greater than the other) could give rise to the same observed count rate, leading to the possibility of incorrect assessments of source activities. On the other hand, in a non-paralyzable system, the observed count rate increases asymptotically to τ^{-1} for true count rates >> τ^{-1} and the system can never be paralyzed. Hence, it is preferred to have a system with non-paralyzable deadtime.

In the dual photon measurement system, the true count rate was determined using the pulser counts registered in the MCA. At low count rates both paralyzable and nonparalyzable systems look similar. From Fig. 3. 4 and 3.5 of the observed versus true count rates of the 42.5 kev and 103 kev photons, it is apparent that the count rates are in such a region. The deadtime of a system is variable; the mean was computed in the above region of count rates. Using the 103 kev count rates in Table 3.1 and the expressions for paralyzable and non-paralyzable systems, a mean value of 8.5 μ sec was obtained for the





The line indicates zero deadtime losses



Fig. 3.5: True vs. observed CPS for 103 kev. The line indicates zero deadtime losses

system deadtime. Since the MCA has a live time clock for deadtime correction of pulses reaching its input stage, it is concluded that the effective deadtime is restricted to that in the detector electronics. Experiments described later, have shown that a count rate less than 2000/s is adequate for DPA in phantom measurements and since the counts lost due to deadtime at this count rate is less than 1.7%, no deadtime corrections are consequently made.

3.3 Self-Absorption

The photon output of the source maybe less than that expected from its activity because of photon absorption within the source material. From the data of Storm and Israel [23], the attenuation coefficients for Sm and O were interpolated to obtain the values for Sm_2O_3 at 42.5 keV and 103 key. The values are shown in Table 3.2. For comparison, the corresponding values for Gd have also been included to indicate the greater fraction of counts absorbed in a Gd source. The theoretical values for Sm_2O_3 attenuation coefficient at 42.5 kev and 103 kev were found to be 4.582 cm²g⁻¹ and 2.221 cm²g⁻¹ respectively. A simple experiment was performed to verify these values. The component B (in fig. 3.1) was removed from an empty graphite capsule and the resultant cavity filled tightly with non-radioactive Sm_2O_3 . Count rate measurements were made with the source and detector in the configuration described in Sec. 3.1 above and with known thicknesses of water as absorber to reduce deadtime errors. In each case, counts were registered in the MCA with and without the second graphite capsule on top of the source in the source holder so that the photons were

.31

| ENERGY | ABSORPTION COEFFICIENT cm ² g ⁻¹ | | | | |
|--------|--|-------|-------|-------|--|
| (Kev) | Sm 0 Sm ₂ 0 ₃ Gd | | | | |
| 42.5 | 5.293 | 0.080 | 4.582 | 5.725 | |
| 103 | 2.570 | 0.008 | 2.221 | 2.748 | |

TABLE 3.2 - Attenuation coefficient for

 $Sm_2 \circ_3$ and Gd

attenuated through known quantities of Sm_2O_3 . The water thickness was different for each measurement so that count rates were of the same magnitude. The thickness (x) of Sm_2O_3 was given by the ratio of the mass of Sm_2O_3 inserted in the capsule and the cross sectional area within the capsule, in units of gcm⁻². This experiment was repeated six times to examine the reproducibility of the measurement. Table 3.3 gives the results obtained from the above experiment. From the mean of the exp(µx) values at 42.5 kev and 103 kev, mean values of the absorption coefficients were obtained as $4.230 \text{ cm}^2\text{g}^{-1}$ and $2.294 \text{ cm}^2\text{g}^{-1}$ respectively. This is in good agreement with the calculated theoretical values. In a typical ¹⁵³Sm source used for DPA in the present study, **39.9%** of photons at 42.5 kev and 16.3% of photons at 103 kev are lost due to self absorption. (34)

For a given source activity the photon output will be greatest when self absorption is minimized. The total intensity of the 42.5 kev and 103 kev photons from 153 Gd is given [30] as 121.6% and 21.1% respectively. From 153 Sm the corresponding values are 60.8% and 28.3%. The apparent advantage of 153 Gd over 153 Sm will be partially offset by the greater self-absorption in 153 Gd. For source masses greater than 0.7 gcm⁻², the 42.5 and 103 kev photon intensities are greater for 153 Sm than for 153 Gd.

3.4 Activity vs. Count Rate

During a period of about six months, five different labelled graphite capsules, each containing about 26 mg of enriched Sm_2O_3 , were

Mass of Sm₂ 0₃ in graphite capsule = 0.4590 g Cross sectional area within capsule = $\pi (\frac{0.635}{2})^2 = 0.3167 \text{ cm}^2$. Area density of Sm₂ 0₃, x = 1.449 g cm⁻² Count time in all following cases = 300 sec.

| ABSOR Sm ₂ ⁰ 3 | BER H ₂ 0 | 42.5 KeV Counts | 103 KeV Counts | Pulser Counts | мх/ _{42.5} | "x/ ₁₀₃ |
|---|-------------------------|--------------------|--------------------------|------------------|---------------------|--------------------|
| Yes No | 50 cc 300 cc | 6249 115116 | 171003 552251 | 17967 17300 | 6.139 | 3.325 |
| No Yes | 300 cc 50 cc | 114317 6165 | 547977 166975 | 17948 17955 | 6.146 | 3.343 |
| Yes No | 50 cc 300 cc | 6212 108120 | 166584 518328 | 17986 17885 | 6.082 | 3.288 |
| No Yes | 250 cc 50 cc | 199282 5798 | 788110 162808 | 17461 17969 | 6.118 | 3.299 |
| Yes No | 50 cc 250 cc | 5779 206571 | 161792 815619 | 17983 17758 | 6.157 | 3.339 |
| No Yes | 250 cc 50 cc | 207047 5903 | 81429 4 160254 | 17495 17974 | 6.138 | 3.350 |
| | | | | Mean | 6.130 | 3.324 |

. Mean μ at 42.5 KeV = 4.230 cm²g⁻¹ ± 0.127 Mean μ at 103 KeV = 2.294 cm²g⁻¹ ± 0.057

TABLE 3.3 - Calculation of source self-absorption

irradiated for a few hours. Count rate measurements were made for the 42.5 kev and 103 kev photons. In every case the source activity was determined using a calibrated Ge(Li) spectrometer. The high energy 463.6 kev, 521.3 kev, (531.4+533.3) kev, 539.1 kev and 609.1 kev photons of ¹⁵³Sm were counted. The intensity data taken from [31] are shown in Table 3.4 along with the detector efficiency at these energies taken from the energy calibration curve. The observed MCA count rates of the 42.5 kev and 103 kev photons at different calculated activities are given in Table 3.5 and plotted in Fig. 3.6 and 3.7 respectively. It is quite evident from these graphs that the observed count rate varies linearly with the source activity. These results enable the use of standard ¹⁵³Sm sources in the dual photon method so that the activity of each source can be derived from the $NaI(T_{\ell})$ detector count rate - (with reference absorbers used to reduce deadtime errors), without having to use the Ge(Li) spectrometer each time. This also facilitates quicker and easier means of dose calculations in invivo applications. In addition, these charts would help determine the useful time period of a given source for DPA.

3.5 Beam Geometry

For stationary, intermittent and continuous scanning measurements the photon beam size at various levels in the absorber is of great importance in DPA. In all these cases narrow beam geometry is preferred in order to minimize errors due to scattering of primary photons in the absorber. The error due to beam size arises from overlapping at the boundaries of the absorber and also due to averaging of attenuation across

| ENERGY | RELATIVE | Ge (Li)DETECTOR |
|--|--|--|
| KeV | INTENSITY | EFFICIENCY |
| 103.1 463.6 521.3 531.4 + 533.3 539.1 609.1 | 100 0.053 0.028 0.357 0.086 0.051 | 3.25×10^{-4} 2.85×10^{-4} 2.80×10^{-4} 2.75×10^{-4} 2.425×10^{-4} |

Absolute intensity of 103.1 KeV photons = 28.3%

<u>TABLE 3.4</u> - Energy-efficiency data for Ge (Li) Spectrometer

| | OBSERVE | D NaI CPS |
|---------------|----------|---------------|
| ACTIVITI M OF | 42.5 KeV | 103 KeV |
| 22.85 | 11686.9 | 6892.8 |
| 17.81 | 9105.4 | 5310.6 |
| 12.38 | 6086.2 | 3304.2 |
| 8.59 | 4209.0 | 2315.0 |
| 4.21 | 1999.5 | 1007.0 |
| 2.98 | 1425.1 | 7 34•3 |
| 2.25 | 1234.3 | 621.0 |
| 1.08 | 861.6 | 442.5 |
| 0.40 | 325.6 | 184.9 |

TABLE 3.5 - Activity vs count rate with NaI (T1)



Fig. 3.6: Observed CPS vs. Activity for 42.5 kev photons. (Y = 0.001979X - 0.0625; r = 0.99)



Fig. 3.7: Observed CPS vs. Activity for 103 kev photons. (Y = 0.003334X + 0.3506; r = 0.99)

the beam profile. According to Watt [32], for greatest accuracy the beam diameter should be about one-third of the smallest bone width that is to be measured with a limitation imposed by counting precision. Typically, in an adult, the lumbar vertebra measures 4.0 cm long, 3.0 cm wide and 2.5 cm thick, and the transverse processes 2.0 cm long, 1.0 cm wide and 0.5 cm thick.

With the 2 mm diameter collimator the beam size at different distances from the source was determined using a strong 153 Sm source at the same distance from the detector (49.6 cm) as before. Photographic films were placed across the beam for a few hours at different distances from the top of the source collimator. After development of these films, the circular images obtained were scanned with a Joyce Loebel scanning microdensitometer. The beam profiles obtained are shown in fig. 3.8. The FWHM was measured in each case to represent the effective beam diameter. The results are given in Table 3.6; a linear relationship between beam diameter and distance from the source collimator face was determined as given below:

Beam Dia. [cm] = Dist. from collimator [cm] x 0.062 + 0.464

For a typical patient lying prone on the scanner table, the beam diameter at entry of the body surface is 0.59 cm and at the exit 1.83 cm. The beam diameter at the detector collimator face is 2.82 cm.

| DISTANCE FROM COLLIMATOR cm. | BEAM DIAMETER cm. |
|---------------------------------|----------------------|
| 5.0 | 0.79 |
| 8.0 | 0.90 |
| 12.7 | 1.27 |
| 17.8 | 1.59 |
| 24.6 | 2.01 |
| 36.5 | 2.70 |

TABLE 3.6 - Photon beam geometry



Fig. 3.8: Beam profiles at different distances from source collimator face. d= distances in cm from source collimator face.

beam width

CHAPTER 4

Dual Photon Absorptiometry

The validity of dual photon transmission measurements of elemental mass was investigated using phantoms of known composition and size. The phantoms tested were aluminum sheets immersed in water and acrylic resin cylinders containing $Ca_3 (PO_4)_2$ again submerged in water. The results obtained with the resin cylinders were compared with measurements using a commercial single photon bone densitometer.

4.1 Theory

As indicated in chapter 1, the equations for transmission of photons at two discrete energies in a two component system are [33]:

$$I = I_0 e^{-(\mu_a m_a + \mu_b m_b)}$$

and

$$I' = I'_{o}e^{-(\mu'_{a}m_{a}+\mu'_{b}m_{b})}$$

Eliminating m_b from these equations, the expression for m_a is given by

$$m_{a} = \frac{\mu_{b}^{\prime} \ln \frac{I}{I} - \mu_{b} \ln \frac{I}{I'}}{\mu_{a} \mu_{b}^{\prime} - \mu_{a}^{\prime} \mu_{b}^{\prime}}$$
$$= \left(\frac{\mu_{b}}{\mu_{b}^{\prime}} \ln \frac{I'}{I_{0}^{\prime}} - \ln \frac{I}{I_{0}^{\prime}}\right) \left(\frac{1}{\mu_{a} - \mu_{a}^{\prime}} \frac{\mu_{b}^{\prime}}{\mu_{b}^{\prime}}\right)$$
(1)

The latter expression facilitates determination of fat composition

in soft tissue in subject measurements. As the fat percent is variable, the ratio $\frac{\mu}{\mu^{T}}$ for soft tissue in a particular individual can be iterated from count rate measurements through points where bone mineral is absent. From this value and theoretical consideration of the ratio of attenuation coefficients, the fat fraction can be evaluated.

The attenuation coefficients at 42.5 kev and 103 kev for the components a (a uminum) and b (water) were interpolated from the data of Hubbell (1969). These values are as follows:

 $\mu_a^{\prime} = 0.5045 \text{ cm}^2\text{g}^{-1}$ at 42.5 kev and $\mu_a = 0.1683 \text{ cm}^2\text{g}^{-1}$ at 103 kev $\mu_b^{\prime} = 0.25548 \text{ cm}^2\text{g}^{-1}$ at 42.5 kev and $\mu_b = 0.16946 \text{ cm}^2\text{g}^{-1}$ at 103 kev

Substituting in equation (1),

$$m_{a} = (0.6633 \ln \frac{I^{42.5}}{I_{0}^{42.5}} - \ln \frac{I^{103}}{I_{0}^{103}})(\frac{-1}{0.1663})$$
(2)

4.2 Absorptiometry in Aluminum

Experiments were performed with aluminum sheets suspended in water. Aluminum thicknesses of 0.1575, 0.325 and 0.65 cm were measured. For each aluminum standard three different water thicknesses were used. The water thickness varied from 7.5 cm to 20.4 cm. Measurements were made with the aluminum sheets located at different sections of the beam, at distances between 5.5 cm and 11.0 cm from the source collimator face. The beam diameter varied from 0.8 cm to 1.15 cm.

In order to reduce errors due to crossover correction and counting statistics, 30% window settings were used at both the 42.5 kev and 103 kev channels in the MCA. The crossover correction measured as described previously was found to be 1.5% with this window setting. The results are presented in Table 4.1. In invivo measurements, patients with different soft tissue thicknesses in the lumbar region will be encountered. In the present experiment water was used analogous to soft tissue; hence different thicknesses were used in the transmission measurements to simulate the conditions of measurements on human subjects. The number of counts accumulated was such that the precision of the results varied from 1.5-45%. From Table 4.1 it can be seen that the accuracy varied from 0.1-3.8%. From the foregoing, it is apparent that within these limits of water thickness and beam size, beam hardening and beam geometry do not affect the results for stationary measurements. The activity of the ¹⁵³Sm source used in these measurements was about 85 mci and source-detector distance was 49.6 cm.

4.3 Absorptiometry Using Bone Phantoms

Bone phantoms in the form of cast acrylic resin cylinders, 25 mm diameter by 75 mm long, were measured by DPA after being immersed in water with their long axes vertical. These phantoms were made to contain concentrations of $Ca_3(PO_4)_2$ varying from 75 mg/ml to 200 mg/ml in increments of 25 mg/ml. The window settings and source-detector distance were maintained the same as with the aluminum standards. The attenuation coefficients at 42.5 kev and 103 kev for the various mixtures were determined using the elemental attenuation coefficients derived from the data of Veigele [35]; the resultant values are shown in Table 4.2.

Three repeat measurements were made for each phantom with the same 153 Sm source (activity = 144 mCi) during a period of two weeks.

| Al | Beam Dia. | Water t | hk.cm | Count | Pulser | Counts | Corr.42 | .5 Kev CPS | 103 KeV | CPS | Measured | True |
|--------|-----------|---------|--------|------------|---------------------|----------------|---------|------------|---------|--------|----------------------|-------------------|
| cm | cm. | with Al | w/o Al | time Sec. | with Al | w/o Al | with Al | w/o Al | with Al | w/o Al | Mass of | Mass |
| | | | | | 1 | | | | | | Al gcm ⁻² | of Al |
| | | | | | | | | | | | | gcm ⁻² |
| | 0.80 | 13.7 | 9.6 | 300 | 1.7616 | 17954 | 290.7 | 952.5 | 1057.9 | 2171.6 | 0.409 | |
| 0.1575 | 0.99 | 11.8 | 7.4 | 500 | 298 0 9 | 292 2 7 | 180.0 | 620.7 | 522.3 | 1109.0 | 0.410 | 0.425 |
| | 1.15 | 10.3 | 8.8 | 500 | <u>29954</u> | 29755 | 123.1 | 296.8 | 357.7 | 598.4 | 0.416 | |
| | 0.80 | 13.7 | 9.6 | 300 | 17962 | 17639 | 322.4 | 1231.0 | 1351.9 | 2849.5 | 0.860 | ļ |
| 0.325 | 0.99 | 10.5 | 7.6 | 500 | <u>29865</u> | 29856 | 122.8 | 407.7 | 382.6 | 733.1 | 0.876 | 0.877 |
| | 1.15 | 10.8 | 8.8 | <u>500</u> | 29901 | 29916 | 94.6 | 286.4 | 321.7 | 580.1 | 0.873 | |
| | 0.80 | 20.4 | 15.3 | 400 | <u>239</u> 88 | 23984 | 34.5 | 274.8 | 443.4 | 1305.7 | 1.782 | |
| 0.65 | 0.99 | 20.4 | 12.7 | 300 | 17875 | 17963 | 34.8 | 529.1 | 438.3 | 1983.2 | 1.778 | 1.755 |
| | 1.15 | 17.8 | 10.2 | 400 | 23791 | 23905 | 53.5 | 834.4 | 558.3 | 2556.3 | 1.808 | |

TABLE 4.1 - Dual photon transmission measurements with Al standards

| PHANTOM NUMBER | CONCN. of Ca ₃ (PO ₄) ₂ IN THE MIXTURE <u>mg</u> ml | [™] 42.5 cm ² g ⁻¹ | / ^M 103 cm ² g ⁻¹ |
|-------------------|---|--|---|
| 2 | 100 | .2811 | .1671 |
| 3 | 175 | .3185 | .1686 |
| 5 | 75 | .2676 | .1655 |
| 6 | 125 | .2932 | .1668 |
| 7 | 150 | .3065 | .1684 |
| 8 | 200 | .3308 | .1700 |

<u>TABLE 4.2</u> - Calculated attenuation coefficients

for bone phantoms

The precision of the resultant mass densities calculated using equation (1), ranged from 1.5-3.5%. Table 4.3 shows the experimental results for the six bone phantoms.

Next, the phantoms were individually scanned on a clinical densitometer (Norland, model 278A) using a single photon source, $^{125}I(28 \text{ kev})$. The cylinders were immersed in 4.0 cm of water with their long axes horizontal and scanned across the diameter. The scanner computations were performed for an edge cut off value of 75%; Table 4.4 gives the observations from three repeat measurements. The mean of the values obtained by DPA (Table 4.3) and the densitometer (Table 4.4) are shown in Table 4.5 and plotted in figure 4.1. A high correlation (r=0.97) was observed between these values.

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| РНАМТОМ | BASELINE | COUNTS/SEC. | ABSORBER QTY. | ABSORBER WIDTH | ABSORBER DENSITY |
|---------|----------|-------------|-------------------|----------------|-------------------|
| No. | Left | Right | gcm ⁻¹ | cm | gcm ⁻² |
| 2 | 1276 | 1266 | 0.452 | 2.121 | 0.213 |
| 2 | 1263 | 1263 | 0.418 | 2.044 | 0.205 |
| 2 | 1289 | 1289 | 0.418 | 2.097 | 0.199 |
| 3 | 1266 | 1266 | 0.766 | 2.320 | 0.330 |
| 3 | 1263 | 1263 | 0.732 | 2.226 | 0.329 |
| 3 | 1285 | 1285 | 0.733 | 2.263 | 0.324 |
| 5 | 1304 | 1276 | 0.333 | 2.047 | 0.163 |
| 5 | 1261 | 1261 | 0.286 | 1.961 | 0.146 |
| 5 | 1285 | 1285 | 0.310 | 2.095 | 0.148 |
| 6 | 1276 | 1276 | 0.528 | 2.079 | 0.254 |
| 6 | 1263 | 1263 | 0.508 | 2.128 | 0.239 |
| 6 | 1261 | 1285 | 0.486 | 2.108 | 0.231 |
| ? | 1289 | 1289 | 0.617 | 2.192 | 0.281 |
| ? | 1261 | 1261 | 0.589 | 2.213 | 0.266 |
| ? | 1289 | 1263 | 0.589 | 2.185 | 0.270 |
| 8 | 1263 | 1263 | 0.880 | 2.321 | 0.379 |
| 8 | 1285 | 1261 | 0.861 | 2.339 | 0.368 |
| 8 | 1285 | 1285 | 0.859 | 2.263 | 0.380 |

TABLE 4.4 - Bone phantom measurements on the clinical densitometer

| ABSORBER | | Count | 42.5 KeV | 103 KeV | Mass |
|--------------------------------------|-------------------|----------------------------|----------------------------------|---|--------------------------------|
| Water cm | Phantom No. | Time Sec. | Counts | Counts | Jensity gm cm ⁻² |
| 12.6 12.6 12.6 12.6 12.6 | - 5 5 5 | 40 40 40 40 | 22625 10424 9941 10017 | 11 4483 71342 69534 69361 | 3.42 3.91 3.28 |
| 12.6 12.6 12.6 12.6 12.6 | - 7 7 7 | 40 40 40 40 40 | 34247 16419 16493 16890 | 95852 7 4713 74440 75410 | 6.83 6.64 6.56 |
| 11.5 12.1 | 8 | 20 20 | 37109 11136 | 105034 72829 | 8.75 |
| 11.5 12.6 | . 8 | 20 20 | 33111 8911 | 94916 60349 | 8.45 |
| 12.6 12.6 | 8 | 30 30 | 33050 11572 | 101292 76347 | ð.36 |
| 12.6 12.6 12.6 12.6 12.6 | - Ŋ.ŊŊ | 110 140 140 140 | 34247 14152 13998 13729 | 95852 75655 75009 74680 | 8.19 8.16 8.35 |
| 11.5 12.1 | 2 | 20 20 | 32208 13644 | 101126 62463 | 4.54 |
| 11.5 12.6 | - 2 | 20 20 | 30316 12765 | 99042 61428 | 4.96 |
| 12.6 12.6 | -2 | 30 30 | 29850 12984 | 96423 60746 | 4.65 |
| 12.6 12.6 12.6 | - 6 6 | 40 40 40 | 34247 19807 19347 | 95852 79259 77929 | 6.25 6.20 |
| 11.5 12.6 | - 5 | 20 20 | 37109 18526 | 105034 79116 | 6.41 |

<u>TABLE 4.3</u> - Dual photon transmission measurements with bone phantoms

| Phantom No. | From DPA gcm ⁻² | From Densitometer gcm -2 |
|----------------|-------------------------------|--------------------------------|
| 2 | 4.71 | 0.206 |
| 3 | 8.23 | 0.328 |
| 5 | 3.53 | 0.152 |
| 6 | 6.28 | 0.241 |
| ? | 6.07 | 0.272 |
| 8 | 8.52 | 0.376 |

TABLE 4.5 - Comparison of results from

DPA and the clinical densitometer



Fig.4.1 : Densitometer vs. DPA measurements (Y = 23.492X + 0.156; r = 0.97)





CHAPTER 5

Conclusions and Suggestions for Further Work

In this work the effectiveness of ¹⁵³Sm as a photon source for DPA was evaluated. This preliminary study has resulted in useful conclusions regarding this source, making way for further invitro and invivo measurements on lumbar vertebrae to proceed.

The 42.5 key and 103 key photons of 153 Sm were found to obey the exponential law of attenuation and thus the fundamental principle of the method is satisfied. The errors due to deadtime were less than 1.7% and hence deadtime correction was not very significant. However, for sources of the order of 1.0 Ci, the deadtime errors would be very high when no absorbers are present. Hence, to determine count rates in air with such sources, known thicknesses of standard absorbers have to be used. In DPA the percentage deadtime error has to be examined with a typical absorber. Self absorption in the samarium source used in DPA was calculated. It was found that 39.9% of the low energy and 16.3% of the high energy photons were absorbed in the source itself. The detector efficiency at 42.5 kev and 103 kev was determined as 0.90 and 0.71 respectively. Considering this and the high percentage loss of photons due to self-absorption in the source, it is suggested that in order to obtain greater count rates (and consequently greater precision) the length of the graphite insert (B in fig. 3.1) be reduced and the capsule irradiated for a longer time. For a desired count rate, the charts of activity vs. count rate can be used to determine the

irradiation time required for a particular source. From the derived photon beam geometry and results of Watt [32], it is concluded that the 2 mm diameter source collimator is suitable for DPA of the lumbar vertebrae.

The dual photon method yielded a precision error of 4.5% and an accuracy error up to 4% in measurements on aluminum standards and bone phantoms immersed in water. These results prove the usefulness of 153 Sm for stationary point measurements by DPA.

In order to measure bone mineral content in the lumbar vertebrae, the femoral neck or the whole body, a rectilinear scan procedure is required. Dunn et al [14] modified an OHIO Nuclear rectilinear scanner for this purpose. They retained the patient table, C-frame and the drive system. The lower detector was replaced by a source holder with a beam shutter and a collimator and the upper detector by a collimated $NaI(T_{\ell})$ detector. The longitudinal movement was controlled by a constant speed stepping motor and the transverse movement by a variable speed stepping motor. Both these motors and the beam shutter were activated by computer generated signals. The positional coordinates of the source-detector system were relayed to the computer by highresolution potentiometers coupled to the motors. As the transverse speed should be uniform over the measurement period, the true speed obtained from the positional information and elapsed time was compared with the input desired speed fed into the computer in order to indicate any deviations. The output from the lower energy in the pulse height analyser was shown on a CRT screen to aid in initial positioning and locating bone edges.

Provided the above modifications were made on our existing scanner the following measurement procedure could be followed for BMC determination. In the first instance, invitro measurements may be performed with the lumbar vertebrae (after removal from a cadaver) immersed in water. The region to be scanned may be descretized into a series of points at one second intervals along each transverse scan path. The scan velocity and scan interval would be set according to the exposure rate and source activity (typically 1.0 ci) to allow for a sufficiently large number of counts to be accumulated during each one second interval in the scan path. For automatic scanning and on-line computation of BMC, the arrangement shown in fig. 5.1 is suggested. The starting and end points of the scan path are predetermined by limit switches on the scanner console. A single control unit may be used to activate the scan start and stop, the beam shutter (on a rack and pinion arrangement) to open and close and the scalers to start and stop accumulation of counts (en route to buffers) in the two energy channels. The buffers are interfaced to a computer and the recorded counts are used for calculation of BMC in each one second interval of the scan path. By means of the potentiometers, the positional information can be transmitted to the computer through ADC's so that the calculated BMC values may be correlated.

The average value of $\frac{\mu}{\mu^{-}}$ defined by equation (1) in Section 4.1 is to be estimated by scanning over regions where bone mineral is absent. This value is subsequently used in the equation to calculate bone mineral mass, m_b, for each accumulated pair of counts. However, in measurements on individuals, this will lead to the determination of





percent fat in soft tissue. The average value of $\frac{\mu}{\mu^{T}}$ is computed by iteration of measurements over areas where bone mineral is absent and this value used to calculate m_b. From the mean value of $\frac{\mu}{\mu^{T}}$ and the linear relation between $\frac{\mu}{\mu^{T}}$ and fat fraction, the percent fat in soft tissue is obtained. The individual values of m_b are summed over the entire scanned region to give the total BMC. The bone width and bone edge of the individual vertebrae may be specified from calibrated cut-off values of m_b at the edges. The density distribution of bone mineral mass can be shown on a CRT screen after intensity modulation.

Before setting on to invivo measurements with human subjects, it is recommended that the radiation dose to points close to the lumbar region in a typical DPA scan be determined using TLD's fixed on an anthropomorphic model. The dose to the skin, ovaries and testicles due to a 1.0 Ci 153 Gd source was measured by Mazess et al [12] and was found to be 2.0, 0.09 and 0.03 mrad respectively in a lumbar spine measurement involving 10 transverse scans over the patient.

The dose to any point on the skin in a typical lumbar scan was calculated for a source strength of 1.0 Ci and scan speed of 5 mm/sec and was found to be about 0.017 mGy. Table 5.2 shows the results of the computation using the following:

$$\dot{D} = \frac{Se^{-\mu r}}{4\pi r^2} \times E \times 1.6 \times 10^{-10} \times (\frac{\mu en}{\rho})_{muscle} \text{ in mGy sec}^{-1}$$

where D is dose rate, S is no. of photons/sec., E is photon energy in MeV, $\frac{\mu_{en}}{\rho}$ is mass-energy absorption coefficient and r (=9.3 cm) is distance from the source.

| kev | (^μ en _ρ) _{muscle} cm ² g ⁻¹ | m Gy sec ⁻¹ |
|---|--|---|
| 40.9 41.5 47.0 69.6 75.4 83.4 89.5 97.4 103.2 | 0.0701 0.0660 0.0572 0.0296 0.0280 0.0263 0.0260 0.0257 0.0256 | 0.0027 0.0047 0.0016 0.0006 0.00002 0.00002 0.00002 0.0001 0.0001 0.0041 |
| • | Total = | 0.01386 |

Scanning speed = 5mm/sec

Beam dia. at entrance to body = 6 mm

 \therefore Total dose at any point = 0.01386 x 1.2

~ 0.017 mGy

Table 5.1: Calculation of dose due to a DPA scan

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